



Nuclear Criticality Safety Division

TRAINING MODULE

CSG-TM-016

COG SOFTWARE



Lawrence Livermore National Laboratory
Livermore, California

Approval History

Revision	Preparer	Reviewer	Approver	Date	Remarks
0	S.T. Huang	D.P. Heinrichs	S.T. Huang	April 30, 1999	Initial version
1	D.P. Heinrichs	A.W. Krass	D.P. Heinrichs	July 9, 2010	Sample problems

Course Plan

Objective

The student will demonstrate a basic level of competency as a COG code user by completing the hands-on criticality safety calculations provided in this training module.

Method of instruction

The instructor will provide each student with a Notebook (with contents specified below) and mentor the student as necessary for the student to successfully complete and execute the sample exercise problems (or equivalent problems).

Requirements

- (1) The student must have access to a computer with a text editor and a postscript viewer (or ps2pdf converter with an Adobe Acrobat Reader). The computer must also have C shell (csh) or TC shell (tcsh) installed with Perl. PC/Windows computers additionally require Cygwin to emulate a LINUX-like environment.
- (2) Prior to scheduling this course, the student must have and obtained the COG installation package from RSICC (or the OECD NEA Databank). Upon request, LLNL may provide user assistance to install the COG code package and Cygwin as required.
- (3) The course duration is approximately 28 hours (3.5 days).

References

- (1) <http://cog.llnl.gov>
- (2) <http://icsbep.inel.gov/handbook.shtml>

Notebook contents

- (1) CSG-TM-016, COG Software
- (2) Mentor Sheet, Training Modules CSG-TM-009, CSG-TM-010, CSG-TM-016, MCNP, SCALE/KENO, and COG Computer Codes
- (3) Evaluation of Training Form
- (4) COG10 software installation README file
- (5) CSAM06-102, Installation and Verification of the COG 10 Code on Unclassified Workstation, Surya
- (6) UCRL-TM-202590, COG – A Multiparticle Monte Carlo Transport Code – User's Manual, Fifth Edition, September 1, 2002
- (7) UCRL-CONF-224715, COG – Publicly Available Now to Criticality Safety Practitioners
- (8) LLNL-PROC-422485, COG – Special Features of Interest to Criticality Safety Practitioners

Course Content

- 1 Installation and verification
 - 1.1 Hardware review
 - 1.2 Operating system review
 - 1.3 The COG website – <http://cog.llnl.gov>
 - 1.4 Obtaining the code from RSICC or the OECD NEA Databank
 - 1.5 Installation – COG10 software installation README file
 - 1.6 Running the verification test problems
 - 1.7 Installation and verification documentation for SQA – CSAM06-102

- 2 Getting started
 - 2.1 The COG manual – UCRL-TM-202590
 - 2.2 **Problem #1** – Jezebel – PU-MET-FAST-001 ————— **Sample Problem**
 - 2.2.1 Review of COG input deck ————— Manual, p. 443
 - 2.2.2 Review of COG data-blocks and keywords — Manual, p. 16
 - 2.2.3 Review of COG comment characters ————— Manual, p. 17
 - 2.3 Running COG
 - 2.3.1 Running interactively in the foreground ————— Manual, p. 10
 - 2.3.2 Running in the background ————— Manual, p. 10
 - 2.3.3 Batch processing on LLNL machines
 - 2.3.4 Killing jobs ————— Manual, p. 10
 - 2.4 Reviewing the sample problem output ————— Manual, p. 445
 - 2.4.1 Review starting random numbers
 - 2.4.2 Review K calculation results
 - 2.4.3 Review fraction of fissions, absorptions, escapes
 - 2.4.4 Review mean times
 - 2.4.5 Review optical paths
 - 2.4.6 Review summary tables
 - 2.4.7 Review restart, timing and misc. memory information

2.5 Additional useful features

- 2.5.1 Add a volume calculation ————— Manual, p. 123
- 2.5.2 Assign colors and make a color picture ——— Manual, p. 112,148
- 2.5.3 Increase the volume calculation resolution ——— Manual, p. 123
- 2.5.4 Discuss boundary conditions and defaults ——— Manual, p. 99

3 Student exercises

- 3.1 **Problem #2** – HEU-MET-FAST-001 ————— **Fissile metal**
- 3.2 **Problem #3** – HEU-MET-FAST-058 ————— **Non-fissile material**
- 3.3 **Problem #4** – U233-SOL-INTER-001 ————— **Fissile solution**
- 3.4 **Problem #5** – LEU-COMP-THERM-033 ————— **Fissile compound**
- 3.5 Instructor completion of Mentor Sheet Qualification Criteria 1:
Modeling fissile and non-fissile materials

4 Introduction to UNITS

- 4.1 Review of the DEFINE UNIT specifications ————— Manual, p. 103
- 4.2 Review of the USE UNIT specifications ————— Manual, p. 105
- 4.3 Review of the FILL specification and default ————— Manual, p. 97
- 4.4 **Problem #6** – HCT002-20 ————— **Complex Unit**
- 4.5 Instructor completion of Mentor Sheet Qualification Criteria 2:
Modeling single unit geometry

5 Finite and infinite and array geometries

- 5.1 Review BOUNDARY conditions ————— Manual, p. 99
- 5.2 **Problem #7** – HEU-COMP-THERM-001-7 ————— **Finite Array**
- 5.3 **Problem #7a** ————— **Interstitial moderator**
- 5.4 **Problem #7b** ————— **Interstitial material**
- 5.5 **Problem #8** – PU-COMP-INTER-001 ————— **Infinite Medium**
- 5.6 **Problem #9** – HEU-COMP-INTER-005 ————— **Infinite Array**
- 5.7 Instructor completion of Mentor Sheet Qualification Criteria 3:
Modeling finite and infinite array geometries

6 Concluding material

- 6.1 Student feedback and completion of “Evaluation of Training” forms
- 6.2 Needs for advanced COG training?
- 6.3 Awarding of course completion certificates

Problem #1

PU-MET-FAST-001-1

Bare Sphere of Plutonium-239 Metal (^{239}Pu Jezebel)

Data block structure

TITLE	PU-MET-FAST-001: JEZEBEL (17.020 kg Pu(95.48)-1.02Ga @ 15.61 g/cc) neutron only calculation with prompt and delayed multiplicities and spectra
BASIC	BASIC neutron delayedn
SURFACES	SURFACES 1 sphere 6.3849 \$ per Section 3.2 length in cm (default) comments
GEOMETRY	GEOMETRY sector 1 alloy -1 boundary vacuum 1 Optional (default vacuum boundary condition) picture cs material -7 0 7 -7 0 -7 7 0 -7 (default resolution w/o titles) volume -7 -7 -7 7 -7 -7 -7 7 -7 14 14 14 "color" not specified (default B&W picture)
CRITICALITY	CRITICALITY npart=5000 nbatch=5005 sdt=0.0001 nfirst=6 norm=1. nsource=1 0. 0. 0.
MIX	MIX Point-wise continuous cross-section library (nlib2 not used) nlib=ENDFB6R7 \$ Atom Densities per Table 3 mat=1 bunches ga 1.3752-3 pu239 3.7047-2 pu240 1.7512-3 pu241 1.1674-4 atoms $b^{-1} \text{ cm}^{-1}$ (1 of 4 options)
	END Any amount of comments may follow the end flag

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Problem #2

HEU-MET-FAST-001-1

Bare, Highly Enriched Uranium Sphere (Godiva)

- 3.1 Model: Solid unreflected sphere (Godiva) of uranium metal.
- 3.2 Dimensions: The radius of the bare sphere is 8.7407 cm.
- 3.3 Materials: The density of the uranium sphere is 18.74 g/cm^3 .
The wt. % of ^{234}U is 1.02 (or 4.9184×10^{-4} atoms/b-cm).
The wt. % of ^{235}U is 93.72 (or 4.4994×10^{-2} atoms/b-cm).
The remainder is ^{238}U (or 2.4984×10^{-3} atoms/b-cm).
- 3.4 Benchmark k-eff: 1.000 ± 0.003 .
- 4.0 Student's COG results:

Problem #3

HEU-MET-FAST-058-1

Highly Enriched Uranium Metal Spheres with Beryllium Reflectors

3.1 Model: Simple one-dimensional spherical-geometry model.

3.2 Dimensions:

Table 18. Benchmark-Model Outer Radial Dimensions.

No.	Be	Ni	Air	HEU	Beryllium
1	0.4983 cm	0.5207 cm	0.5555 cm	5.1805 cm	25.4497 cm

3.3 Materials:

Table 19. Atom Densities.

Material	Isotope or Element	Composition (wt.%)	Mass (grams)	Atom Density (atoms/barn-cm)
Be (internal) (1.7751 g/cm ³)	Be	100.00	0.92	1.1862 x 10 ⁻¹
Ni (8.90 g/cm ³)	Ni	100.00	0.65	9.1290 x 10 ⁻²
Air (1.2046 x 10 ⁻³ g/cm ³)	N	74.521	--	3.8595 x 10 ⁻⁵
	O	22.906	--	1.0386 x 10 ⁻⁵
	Ar	2.542	--	4.6160 x 10 ⁻⁷
	C	0.027	--	1.6307 x 10 ⁻⁸
	Rare Gases	0.004	--	--
HEU (Core No. 1) (18.5075 g/cm ³)	²³⁴ U	1.00	107.65	4.7621 x 10 ⁻⁴
	²³⁵ U	93.17	10,029.79	4.4179 x 10 ⁻²
	²³⁸ U	5.83	627.60	2.7295 x 10 ⁻³
	HEU	100.00	10,765.04	--
Be (outer) (1.84 g/cm ³)	Be	100.00	--	1.2295 x 10 ⁻¹

3.4 Benchmark k-eff: 1.0000 ± 0.0026.

4.0 Student's COG results:

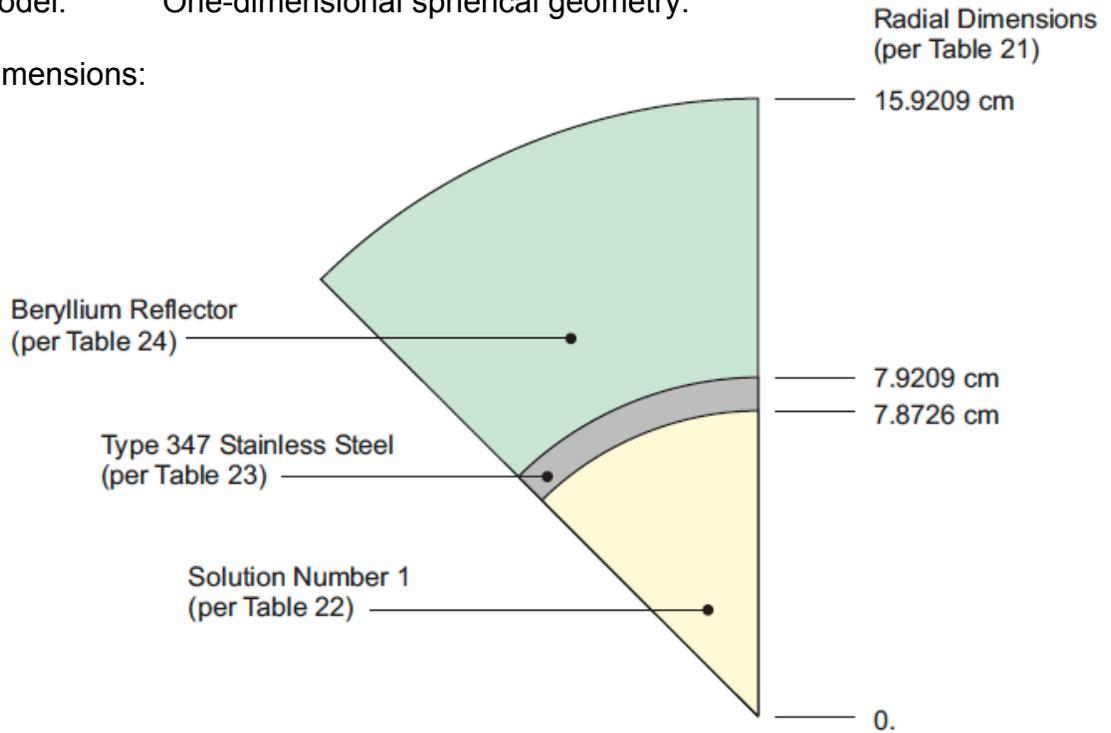
Problem #4

U233-SOL-INTER-001-1

Uranyl-Fluoride (²³³U) Solutions in Spherical Stainless Steel Vessels with Reflectors of Be, CH₂ and Be-CH₂ Composites – Part I

3.1 Model: One-dimensional spherical geometry.

3.2 Dimensions:



3.3 Materials:

²³³U Solution No. 1

Solution →	No. 1
Isotope or Nuclide	Atom
²³² U	4.5608×10^{-8}
²³³ U	2.2379×10^{-3}
²³⁴ U	2.4316×10^{-5}
²³⁵ U	8.9598×10^{-7}
²³⁸ U	7.1284×10^{-6}
H	5.5183×10^{-2}
O	3.2043×10^{-2}
F	4.7182×10^{-3}

SS Type 347 at 8.0 g/cc

Nuclide	Composition (wt.%)	Atom Density (atoms/barn-cm)
Fe	71	6.1248×10^{22}
Cr	18	1.6678×10^{22}
Ni	11	9.0264×10^{21}

Beryllium at 1.82 g/cc

Material	Nuclide	Atom Density (atoms/barn-cm)
Beryllium	Be	1.2161×10^{23}

3.4 Benchmark k-eff: 1.0000 ± 0.0083 .

4.0 Student's COG results:

Problem #5

LEU-COMP-THERM-033-24

Reflected and Unreflected Assemblies of 2 and 3%-Enriched Uranium Fluoride in Paraffin

3.1 Model: Simple three-dimensional cuboid model.

3.2 Dimensions: 76.65 x 76.65 x 78.08 cm³

3.3 Materials:

Table 10. Atom Densities (atoms/barn-cm) for UF₄-Paraffin Fuel Mixtures.

Isotopes → Mixture ↓	²³⁵ U	²³⁸ U	²³⁴ U	H	C	F
U(2)F ₄ -1	1.5799E-04	7.6424E-03	1.5867E-06	3.0908E-02	1.4860E-02	3.1208E-02

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3.4 Benchmark k-eff: 1.0000 ± 0.0040.

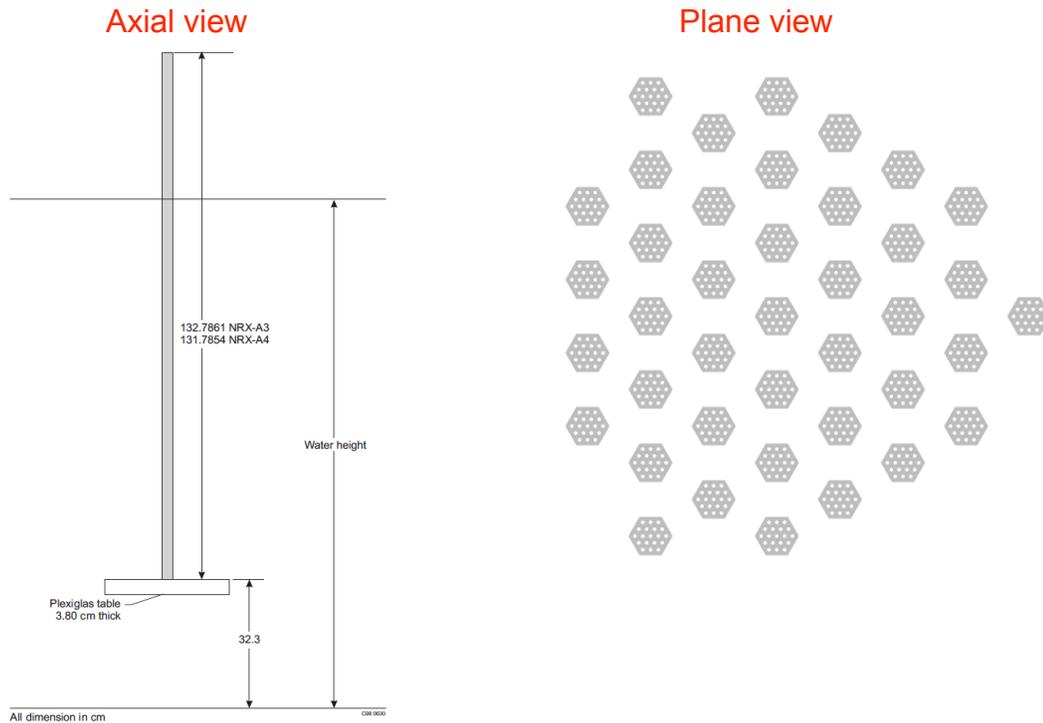
4.0 Student's COG results:

Problem #6

HEU-COMP-THERM-002-20

Graphite and Water Moderated NRX-A3 and NRX-A4 Assemblies

- 3.1 Model: Plane array of 40 NRX-A4 elements in water.
- 3.2 Dimensions: Critical water height of 162.9 cm.
 Plexiglas table is 58 x 58 x 3.80 cm³.
 Lattice pitch of the NRX-A4 elements is 3.73634-cm.
 Effectively infinite water lateral reflection.
 Each NRX-A4 element measures 1.91516-cm “across-the-flats” and has 19 (0.25146-cm diameter) holes on a 0.41048-cm pitch.



- 3.3 Materials:

Nuclide	Atoms/barn-cm
	NRX-A4
²³⁵ U	1.0716×10^{-3}
²³⁸ U	7.7806×10^{-5}
C	9.3228×10^{-2}

Material	Nuclide	Atoms/barn-cm
Water (20°C, air-free, 0.9982 g/cm ³)	H	6.6735×10^{-2}
	O	3.3368×10^{-2}
Plexiglas ($\rho=1.18$ g/cm ³)	H	5.6782×10^{-2}
	C	3.5489×10^{-2}
	O	1.4196×10^{-2}

- 3.4 Benchmark k-eff: 1.0020 ± 0.0043 .

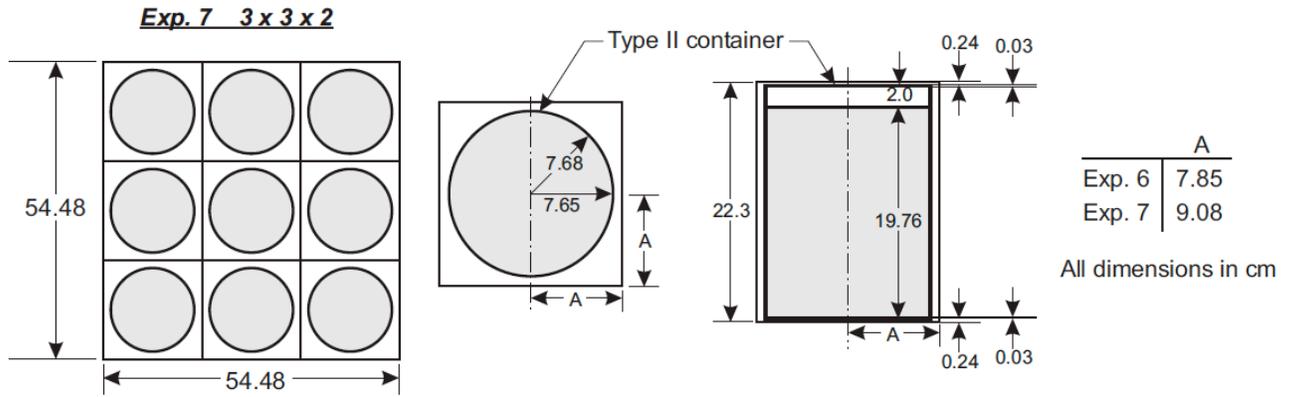
- 4.0 Student's COG results:

Problem #7

HEU-COMP-MIX-001-7

Arrays of Cans of Highly Enriched Uranium Dioxide Reflected by Polyethylene

- 3.1 Model: 6"-polyethylene reflected 3x3x2 array of Type II cans
- 3.2 Dimensions: 2.80 cm horizontal gaps between units
0.24 cm gap between CH₂ reflector and tops and bottoms of cans
0.48 cm gap between stacked cans



All arrays surrounded by 15.24 cm polyethylene in 3 dimensions (not shown).

UO₂ at 5.505(0.998949) g/cm³
 0.87799 gU/gUO₂
 0.9911 g²³⁴U/gU
 93.1528 g²³⁵U/gU
 0.3367 g²³⁶U/gU
 5.5194 g²³⁸U/gU

3.3 Materials:

CH₂ at 0.92 g/cm³

Material	atoms/cm-barn
Polyethylene	
H	7.8996 x 10 ⁻²
C	3.9498 x 10 ⁻²

Fe-1C at 8.45 g/cm³

Material	atoms/barn-cm
Carbon Steel Can ^(a)	
Fe	9.0207 x 10 ⁻²
C	4.2367 x 10 ⁻⁵
UO₂ Contents	
U ²³⁴	1.2313 x 10 ⁻⁴
U ²³⁵	1.1524 x 10 ⁻²
U ²³⁶	4.1476 x 10 ⁻⁵
U ²³⁸	6.7418 x 10 ⁻⁴
O	2.5255 x 10 ⁻²

3.4 Benchmark k-eff: 0.9997 ± 0.0038.

4.0 Student's COG results:

(a) Based on density of 8.45 g/cm³.

Problem #7-a

Exercise: Add interstitial water outside and in-between Type II cans (flooding).

Student's COG results:

Problem #7-b

Exercise: Add interstitial alumina (Al_2O_3) outside and in-between Type II cans.

Student's COG results:

Student's remarks on the pros and cons of using "SECTOR" versus "FILL" to model the interstitial materials:

Problem #8

PU-COMP-INTER-001-1

K-infinity Experiments in Intermediate Neutron Spectra for ^{239}Pu

3.1 Model: A single-material infinite (homogeneous) medium.

3.2 Dimensions: Model as a “box” with reflecting boundary conditions.

3.3 Materials:

Homogeneous Plutonium/Boron/Graphite Null-Reactivity Sample	H	1.077×10^{-4}
	$^{10}\text{B}^{(a)}$	1.0151×10^{-4}
	^{11}B	4.0859×10^{-4}
	C	7.090×10^{-2}
	O	2.707×10^{-3}
	Ca	8.280×10^{-4}
	^{239}Pu	2.735×10^{-4}
	^{240}Pu	1.549×10^{-5}
^{241}Pu	1.072×10^{-6}	
^{242}Pu	5.800×10^{-8}	

3.4 Benchmark k-inf: 1.0000 ± 0.0110 .

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4.0 Student's COG results:

Problem #9

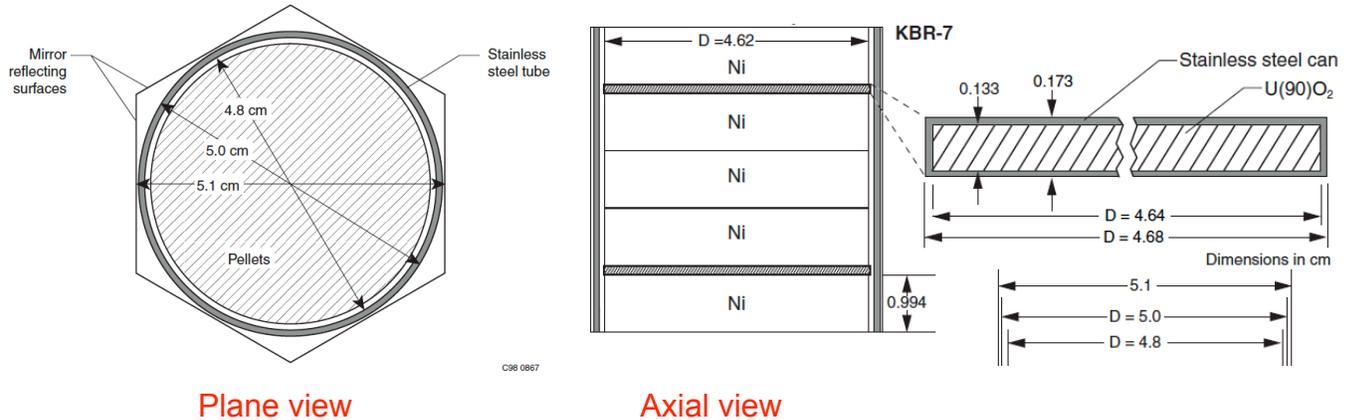
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HEU-COMP-INTER-005 (KBR-7)

K-infinity Experiments in Intermediate Neutron Spectra for Various Structural Materials

3.1 Model: Infinite lattice of hexagonal-pitched cells.

3.2 Dimensions: 5.1-cm pitch



3.3 Materials:

Pellet or Tube	Density (g/cm ³)	Nuclide	Atomic Density
Stainless Steel Tube	8.13314	Fe	6.1216E-02
		Cr	1.6579E-02
		Ni	8.5122E-03
		Mn	1.0698E-03
		Ti	6.1377E-04
		Si	8.7196E-04
		C	4.0778E-04
Uranium Dioxide Pellet	6.6698	²³⁵ U	1.3536E-02
		²³⁸ U	1.4663E-03
		O	3.0377E-02
Can of the Uranium Dioxide Pellet	7.9502	Fe	5.9839E-02
		Cr	1.6206E-02
		Ni	8.3207E-03
		Mn	1.0458E-03
		Ti	5.9996E-04
		Si	8.5234E-04
		C	3.9861E-04
Nickel Pellet	8.8698	Ni	9.0921E-02
		Co	9.0636E-05

3.4 Benchmark k-eff: 1.032 ± 0.004.

4.0 Student's COG results:

